$NEACRP-L-299$

SANDIA REPORT

 $SAND88-0380$ · TTC-0783 · UC-71 Unlimited Release Printed August 1988

Standard Thermal Problem Set for the Evaluation of Heat Transfer Codes Used in the Assessment of Transportation Packages

R. E. Glass, Sandia National Laboratories, United States; M. Burgess, Atomic Energy Establishment, Winfrith, United Kingdom; E. Livesey, British Nuclear Fuels, PLC, United Kingdom; J. Geffroy and S. Bourdon, Commissariat A L'Energie Atomique, France; D. Mennerdahl, E. M. Systems, Sweden; A. Cherubini and S. Giambuzzi, ENEA, Italy; P. Nagel, Nuclear Energy Agency, Organization of Ecomonic Cooperation and Development

 \sim

 \mathcal{L}

,,

93160001

Prepared by
Sandia National Laboratories
Albuquerque, New Mexico 87185 and Livermore, California 94550 for the United States Department of Energy under Contract DE-AC04-76DP00789

SF2900Q(8-81)

,,

Issued by Sandia National Laboratories, operated for the United States Department of Energy by Sandia Corporation.

NOTICE: This report was prepared as an account of work sponsored by an agency of the United States Government. Neither the United States Gover ment nor any agency thereof, nor any of their employees, nor any of their contractors, subcontractors, or their employees, makes any warranty, express or implied, cr assumes any legal liability or responsibility for the accuracy completeness, or usefulness of any information, apparatus, product or process disclosed, or represents that its use would not infringe privately owned rights. Reference herein to any specific commercial product, process, or service by trade name, trademark, manufacturer, or otherwise, does not necessaril constitute or imply its endorsement, recommendation, or favoring by the United States Government, any agency thereof or any of their contractors or subcontractors. The views and opinions expressed herein do not necessarily state or reflect those of the United States Government, any agency thereof or any of their contractors.

go Ruulie

Printed in the United States of America Available from National Technical Information Service U.S. Department of Commerc 5285 Port Rc,yal Road Springfield, VA 22161

NTIS price codes Printed copy: A03 Microfiche copy: A01

 $NEACRP-L-299$

93160003

Distribution Category UC-71

SAND88-0380 TTC-0783 Unlimited Release

STANDARD THERMAL PROBLEM SET FOR THE EVALUATION OF HEAT TRANSFER CODES USED IN THE ASSESSMENT OF TRANSPORTATION PACKAGES

R. E. Glass* Sandia National Laboratories** United States

M. Burgess Atomic Energy Establishment, Winfrith United Kingdom

> E. Livesey British Nuclear Fuels, PLC United Kingdom

J. Geffroy' S. Bourdon Commissariat A L'Energie Atomique France

D. Mennerdahl

E. M. Systems

Sweden

A. Cherubini S. Giambuzzi

ENEA Italy

P. Nagel

Nuclear Energy Agency Nuclear Energy Agency Corpentration of Pronomic

*This work performed at Sandia National Laboratories, Albuquerque, New * This work performed at Sandia National Laboratories, Albuquerque, New Mexico, supported by the U.S. Department of Energy under Contract DE-AC04-76DP00789

**A United States Department of Energy Facility

ABSTRACT

The design and certification of packages used for the shipment of nuclear materials requires that well established analytical techniques are available to predict the behavior of the packagings during normal transport and hypothetical accident environments. The Organization of Economic Cooperation and Development (OECD), through the Nuclear Energy Agency's Committee on Reactor Physics (NEACRP), has established specialists' meetings in the areas of heat transfer, criticality, and shielding. The subject of this paper is the work resulting from the Specialists' Meetings on the Heat Transfer Assessment of Transportation Packages.

The heat transfer group held its first meeting in April 1985 with the purpose of defining a set of thermal benchmark problems and providing solutions. A set of six two-dimensional thermal problems has been defined and analytical solutions provided. These problems, and the corresponding solutions, are provided in this report. In addition, magnetic tapes, including computer code input and output, are being provided to the NEA Data Bank.

The codes used to provide numerical analysis range from the cask specific codes such as RIGG to general purpose codes such as Q/TRAN. These codes were all used in the two dimensional mode although many were capable of solving three dimensional problems.

 7054 -i-

PARTICIPATING ORGANIZATIONS

- ${\tt AEEW}$ Atomic Energy Establishment, Winfrith (United Kingdom)
- **BNFL** British Nuclear Fuel, PLC (United Kingdom)
- CEA Commissariat A L'Energie Atomique (France)
- EMS E M Systems (Sweden)
- **ENEA** Comitano Nazionale Per La Ricerca E Per Lo Sviluppo Dell'Energia Nucleare E Delle Energie Alternative (Italy)
- ORNL Oak Ridge National Laboratory (United States)
- SNL Sandia National Laboratories (United States)

Do JJJJJJ

TABLE OF CONTENTS

 \cdot

 $\hat{\mathcal{A}}$

 $\frac{1}{2} \left(\frac{1}{2} \right)$

 \sim

ILLUSTRATIONS

Page

l

 $\overline{}$

 $\tilde{\mathcal{A}}$

TABLES

NOMENCLATURE

 $\hat{\boldsymbol{\epsilon}}$

 \overline{a}

 $\ddot{}$

Se Audit

Introduction

During a preparatory meeting in Paris on June 21 to.22, 1979, several Nuclear Energy Agency (NFA) member countries expressed interest in exchanging information and experience on various aspects of spent fuel transportation cask design. As a result of this meeting, working groups were established under the auspices of the Committee on Reactor Physics (CRP) in rhe areas of heat transfer, criticality, and shielding.

The heat transfer group was established to define a set of cask-like thermal problems and to provide solutions. The problem set and its solutions are available to benchmark numerical codes.

The problems are designated according to the proposing member (France, FR; United Kingdom, UK; and United States, US) and problem number. Hence, the problems are FR-1, UK-1, UK-2, UK-3, US-1, and US-2. During the shipment of spent fuel, numerous thermal transport mechanisms are occurring simultaneously. All spent fuel casks have a heat source (fuel) which rejects heat through a liquid or gaseous medium to the cask wall. The heat is then conducted through the cask wall and rejected at the surface through a combination of convection and radiation. During a fire, the transport of heat is reversed with the greater heat source being on the outside of the cask, and the same heat transport mechanisms then work to transport heat in towards the fuel.

The problems that have been defined address each of these areas. UK-1 is a simulated horizontal fuel pin array in a gas environment. FR-1 addresses the situation where the fuel is surrounded by sodium which is allowed to undergo phase changes. UK-3 addresses the potential for thermal stratification and pressure buildup in a water-filled cask. US-1 simulates a heat source with conduction through the cask wall and heat rejection by convection at the cask surface. UK-2 simulates heat rejection by fins. US-2 is a multiple layered cask in a fire environment with e thermal shield. This configuration involves a two-dimensional radiation analysis.

Description of Thermal Codes

The thermal codes used in the intercomparison for each problem were selected by the user. This results in different codes being used for each problem. These codes range from those developed for a specific purpose, such as fuel pin simulation (RIGG) to the large multipurpose heat transfer code (Q/TRAN).

The selection of codes used indicates that a large number of thermal codes are available to select from and that a given problem can be solved using a variety of tools. This makes a standard problem set particularly valuable in evaluating the available codes.

The codes used in this exercise are summarized in Table I. This table presents the advertised capabilities of each of the codes. The geometry section addresses the number of dimensions and coordinates systems that the codes can handle. In the standard problem set, only one- and twodimensional problems are presented for ease in modeling, although many of the codes are capable of solving the three-dimensional problems that arise in practice.

The temporal section addresses whether the codes solve steady state or transient problems and further whether they use an explicit or implicit integration technique in providing the transient solutions. The ability to solve steady state problems directly, as opposed to converging a transient solution, is significant to the cost of providing solutions. This is most applicable to solving normal transport problems or in establishing the initial temperature distribution prior to a thermal transient, such as exposure for 30 minutes to an 800°C ambient. The explicit versus implicit technique is of interest as it affects the stability and efficiency of the solution. An explicit integration technique solves the heat transfer equation at a time, $t + \Delta t$, based only on solutions at the previous time step, t. This technique is conditionally stable and hence requires small time steps. An implicit integration technique uses the equilibrium conditions at time, $t + \Delta t$. This technique is unconditionally stable and hence allows larger time steps.

 $\sqrt{2}$, $\sqrt{3}$

 7054 -2-

Table I: Code Matrix

Geometry	HEAT6	HEAT5	TRUMP	TEMPEST	TAC-2D	COYOTE	Q/TRAN	SINDA	TAU	RICG	TLUFT	KIRLFINE	\prec COBRA
$1 - D$	Y	Y	Y	Y	$\mathbf Y$	Y	Y	Y	Y	N	Y		
$2 - D$	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	N	Y	Y
$3-D$	Y	Y	Y	Y	N	N	Y	Y	Y	N	N	Y	N
Cartesian	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y
Cylindrical	Y	Y	Y	Y	Y	Y	Y	Y	Y	N	N	Y	N
Irregular	N	N	Y	N	N	Y	Y	Y	Y	N	N	Y	N
Temporal													
Steady State	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y
Transient Implicit	Y	Y	Y	Y	Y	Y	Y	Y	Y	N	Y	Y	N
Transient Explicit	Y	Y	Y	Y	Y	Y	Y	Y	N	N	N	$\mathbf N$	Y
Physics													
Conduction	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	N
Radiation	Y	N	Y	Y	N	N	Y	Y	Y	Y	Y.	Y	Y
Heat Generation	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	N	Y	Y
Variable Properties	Y	Y	Y	Y	Y	Y	\mathbf{Y}	Y	Y	Y	\blacksquare	Y	$\mathbf N$
Phase Change	Y	Y	Y	N	N	N	Y	Y	N	N	N	Y	N
<u>Type</u>													
Finite Element Method	N	N	N	N	N	Y	N	N	Y	N	N	Y	N
Finite Difference Method	Y	Y	Y	Y	Y	N	N	Y	N	Y	Y	N	Y
Thermal Network Analogy	N	$\mathbf N$	N	$\mathbf N$	N	N	Y	N	N	N	N	N	N
Boundary Conditions													
Transient	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	N
Temperature	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	\overline{N}
Heat Flux	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	N
Convection	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y
Radiation	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y
Calculation of View Factors	N	${\bf N}$	N	N	N	N	N	N	Y	Y	Y	N	Y

 \cdot

The boundary condition section addresses whether a code can address problems with a variety of boundary conditions, such as fixed temperature, heat flux, convection, and radiation. This identifies the type of problem that can be solved and what approximations must be made in simulating the actual boundary 'conditions.

The section on physics identifies the physical phenomena that can be simulated with the code. These include the basic heat transfer phenomena of conduction, convection, and radiation as well as heat generation, phase change, and variable properties. There are additional fluids-related capabilities, such as phase change with convection currents or volume change, which are not addressed because they are either not generally used or are particular to a specific cask.

The final section specifies the type of code. These are finite difference, finite element, and thermal network analogy. This information is often needed to select pre- and post-processors and as an indication of ease of using with the codes.

US-l: Internal Heat Source

US-1 consists of the two-region cylindrical geometry shown in the cross sectional view provided in Figure 1. The interior region (Region I) contains a volumetric heat source with an energy generation rate representative of the internal decay heat of single 120-day-old Pressurized Water Reactor (PWR) spent fuel assembly. The internally generated heat is removed at the outer surface of the cylinder by convective cooling to the environment.

The geometrical and thermal specifications for US-1 are provided in Table II. The thermal properties were selected for calculational purposes and are not representative of cask designs or materials.

This problem has an exact analytical solution for comparison with the numerical results.

 7054 -4.

8

Figure 1. US 1--Cask with Internal Heat Source

Table II: US-1 Thermal Characteristics

The codes used in this solution include SINDA (SNL); HEATING-6 (EMS, ENEA, ORNL); Q/TRAN (SNL); I)ELFINE (cEA); AND TAU (AEEW, BNFL).

The exact and numerical solutions reported to 1° accuracy are given in Table III and the graphical solution reported to 0.1° accuracy in Figure 2. The numerical and exact solutions agreed within reported accuracy.

Table III: US-1 Results

These results indicate that a broad spectrum of codes are available for performing problems involving an internal heat source with specified convective cooling at the surface.

.
: أكد المعام

Figure 2. US-1--Temperature Versus Normalized Radial Position

 -1

 \mathcal{A}^{\prime}

91 DOP 96

US-2: Cask with Annular Regions and Shield

US-2 is based on a cask configuration consisting of several different annular regions. The multiregion geometry for US-2 is shown in Figure 3, while the thermal and geometric description of each region is provided in Table IV. Region I contains a volumetric heat source simulating the decay heat of a spent fuel assembly. Region II is a monolithic steel wall providing gamma shielding and structural integrity. Region III is considered to be a voided neutron shield. Region IV is the outer wall of the neutron shield. The single mode of heat transfer between Regions II and IV is thermal radiation. The cask/shield arrangement is assumed to transfer heat to the surrounding environment by thermal radiation only. Since the area between the cask and shield is transparent to thermal radiation, there is also a thermal radiation exchange between the bottom of the cask and the upper surface of the shield.

A three-part solution was generated for US-2. Those three parts consist of: i) a steady state solution to define initial conditions generated during normal transport, ii) a 30-minute fire transient with an environment temperature of 8OO"C, and iii) a cool-down period in an ambient environment for 60 minutes duration.

The tabular results are given in Table V, and the mean value graphical results are given in Figure 4.

The largest deviation from the mean temperatures was 15°C at Location T3 after 90 minutes. The codes used in analyzing this problem were SINDA (SNL), Q/TRAN (SNL), TAU (AEEW, BNFL), DELFINE (CEA), and HEATING-6 (ORNL, ENEA, EMS).

This same problem, when analyzed with only one-dimensional radiation models, produced a 50°C temperature deviation at T3, where the cask body faces the shield. This is due to the lack of radiant energy interacting with the cask from the fire.

Figure 3. US-2--Cask with Annular Regions and Shield

Ambient Environment, Steady-state: $t = 0$ $T_{\infty} = 54.4^{\circ}C$ Fire Environment, Transient: $0 < t < 30$ min $T_{\infty} = 800^{\circ}C$ Ambient Environment, Transient: $30 < t < 90$ min $T_{\infty} = 54.4^{\circ}C$

a

e

.
25 - Jane

Radiation Properties

 $\varepsilon = \alpha = 1$ (All surfaces and environment considered black)

Table V: US-2 Results - Temperatures at Specified Locations

FR-1: Transport in Sodium

The proposed model, FR-1, as shown in Figure 5 is taken from the transport method used in France to ship the "monitored" fuel pin assemblies from Super Phenix to laboratories for analysis.

The goal of the proposed model is to verify, on a simplified geometry, the validity of the numerical formulation implemented in the codes which take into account phase change phenomena.

The geometry and thermal specifications are provided in Table VI. The model consists of a radial section of a cask containing a sheath filled with sodium in which the irradiated assembly is placed. The residual power is dissipated to the environment through a finned surface.

In the initial state the sodium is completely solidified. The calculation is then performed in a transient state where the cask is subjected to a temperature of 800°C.

Two variations of this problem were analyzed. The first variation (FR-la) included a steady-state solution to determine the initial condition followed by a l-hour transient where the cask is subjected to an 800°C ambient environment. These conditions are given in Table VII. The temperature dependent conductivity of air is given in Table VIII. This problem is solved using the codes HEATING-6 (EMS, ENEA); TAU (AEEW, BNFL); and DELFINE (CEA). The temperature histories are given in Figure 6.

 $95 - 119$

Figure 4. US-2--Temperature Versus Time

7054

 $-12-$

 $\frac{1}{2}$

Figure 5. FR-1--Transport in Sodium

Table VI: FR-1 Thermal Characteristics

Table VII: FR-la Initial and Boundary Conditions for One-Hour Transient

Steady State

Time, 0 Ambient temperature, C"C Global heat transfer coefficient (convection + radiation), 20 W/m^2 °C Surface emissivity, 0.5 Heat transfer in annular region between the sheath and container is by radiation and conduction in air

Transient

Time, O-60 minutes Ambient temperature, 800°C Global heat transfer coefficient, 200 W/m^{2°}C

 $-14-$

 $\frac{1}{2}\left|\frac{d}{dt}\right|^{2}=\frac{1}{2}\left|\frac{d}{dt}\right|^{2}$

Table VIII: Temperature Dependent Conductivity of Air

The second variation assumed a 30-minute transient as defined in Table IX followed by a l-hour cool down. The solutions to this problem were obtained using HEATING-6 (ORNL) and SINDA (SNL) and are given in Figure 7.

Table IX: FR-lb Initial and Boundary Conditions for 30-Minute Transient

Steady State

Time, 0 Ambient temperature, 0°C Global heat transfer coefficient 30 W/m2"C Surface emissivity, 0.5 Heat transfer in annular region between the sheath and container is by radiation and conduction in air

Transient

Time, O-30 minutes Ambient temperature, 800°C Global heat transfer coefficient, 200 $W/m^{2\circ}C$

Cool Down

Time, 30-90 minutes Ambient temperature, 0°C Global heat transfer coefficient, 30 W/m^2 °C

 95.501

Figure 6. FR-la Temperatures Versus Time

Figure 7. FR-1b Temperatures Versus Time

98000196

 $-LI$ -

The maximum variation from the mean temperature for each of these problems was 2.3"C at T4 at 3,600 seconds. This indicates that even with phase change, the codes are able to produce consistent results.

UK-l: Irradiated Fuel Element in a Gas Environment

UK-l represents a simulated PWR fuel element in a gas environment. Figure 8 shows the 16 x 16 array of heated and unheated pins simulating the fuel and control rod positions. The array is contained in an isothermal enclosure. The internally generated heat is removed by conduction and radiation to the internal surface of the enclosure.

The dimensions and material properties are given in Table X. This problem is based on an experiment performed by BNFL. The steady state analytical and experimental results are given in Figures 9 and 10 where analytical results are enveloped by the shaded region. The codes used to perform the analyses were HEATING-6 (ORNL); RIGG (AEEW, BNFL); COBRA (CEA); and Q/TRAN (SNL).

The greatest deviation between experiment and the analytical envelop was 6 percent. The largest absolute variation in the analytical solutions was 20°C at the array center. This represents a 6 percent variation in analytical results but only 4 percent maximum variation from experiment, These figures indicate that the analytical results are consistent and in good agreement with experimental data.

UK-2: Plane Finned Surface

UK-2, shown in Figure 11, represents a plane surface with a uniform array of parallel rectangular fins attached. The problem represents three phases in a fire test. The first is the pretest, steady state condition where heat is transferred by natural convection from an internal fluid at a fixed temperature to the plane inside wall. Heat is conducted through the wall and dissipated by radiation and natural convection from the outside wall and fin surfaces to constant temperature surroundings.

 7054 -18.

Figure 8. UK-1 Irradiated Fuel Element in a Gas Environment

Sansad2N

Geometry and Dimensions

The second phase is the fire transient where heat is supplied by radiation and forced convection from a hot external fluid. After conduction through the fins and the body, it is rejected by natural convection to the internal fluid. The third phase is the cool down period where heat absorbed during the fire transient is rejected to the surroundings by the same process as used to derive the initial steady state condition.

Two magnitudes of surface emissivity are considered to assess the ability of the calculation methods to treat heat transfer between reflecting surfaces. The dimensions and material properties are given in Table XI, and boundary conditions are given in Table XII.

 $\frac{3}{2}$

 \mathbf{t}

7054

 $-51 -$

Figure 10. UK-1 Temperatures Along Line A-C

Figure 11. UK-2 Plane Finned Surface

Soludade

Table XI: UK-2 Dimensions and Material Properties

The codes used to analyze this problem were TAU (AEEW, BNFL); HEATING-6 (ORNL, ENEA, EMS); Q/TRAN (SNL); and DELFINE (CEA).

The results are given in Figures 12, 13, and 14. The plots are all from the results of the TAU code with standard deviations and greatest deviations derived from the other analyses indicated. Figure 12 shows the comparison for the fin tip temperature of the results for the two emissivities. Figure 13 shows the fin root and internal surface temperatures. The temperature distribution around the fin perimeter is given in Figure 14.

The steady state temperatures shown in Table XIII are very closely spaced, If the different results are averaged, then the spread, represented by a standard deviation, is at most 0.3"C at the fin root.

 \bullet

 \bullet

an
Maria Martin Barat (1970)

At the end of the fire transient (i.e., 30 minutes), the standard deviation on the mean has increased to 6.6° C at the fin root. This is, however, only 5 percent of the temperature rise and, thus, an acceptable accuracy for most purposes.

At the end of the cool down period, all temperatures are approaching the initial, steady state values so the scatter is again small. The standard deviation is less than 1°C (fin tip) or 2 percent of the difference from ambient. At this point, there is little difference between the temperatures predicted using the two different emissivities.

(a) Initial Steady State Conditions

External - Ambient Temperature $T_a = 38^{\circ}C$ Environment Emissivity $\varepsilon_a = 1.0$ Heat Transfer Coefficient $h_a = 2.08 s^{1/3} w/m^2K$ $\theta_{\rm S} = T_{\rm S} - T_{\rm a} K$ T_S = local surface temperature Surface Emissivity $\varepsilon_{\rm S} = 1.0$ and 0.8 Radiation exchange between all surfaces with finite view factors (fin-to-fin, fin-to-root area). Gas within the fin cavity does not absorb, emit, or scatter radiation.

Internal - Fluid Temperature $T_w = 100^{\circ}C$ Heat Transfer Coefficient $h_w = 500 \theta_i^{1/3} w/m^2K$ $\theta_i = T_i - T_w K$ $T_W =$ inside wall temperature.

(b) Fire Test Transient Conditions - Duration 30 Minutes

Initial temperatures from (a) above

External - Ambient Temperature $T_a = 800^{\circ}$
Environment Emissivity $\varepsilon_a = 1$ Environment Emissivity $\varepsilon_a = 1.0$ Heat Transfer Coefficient $h_a = 10.0 \text{ w/m}^2\text{K}$ Surface Emissivity $\varepsilon_a = 1.0$ and 0.8 Otherwise as (a) above.

Internal - as for (a) above.

(c) Cool Down Transient Conditions - Duration 60 Minutes .Initial temperatures from end of transient (b) above External and internal boundary conditions as for (a) above.

An important temperature for a wet flask application is the internal surface temperature at the end of the fire transient. This, in the example considered here, is very close to the peak internal temperature and would yield the highest vapor pressure contribution to internal pressure. this case the average temperatures are about 34°C above the internal water temperature, and the l.l"C standard deviations corresponds to 3 percent on this temperature difference.

7054

 $-36 -$

 $\frac{1}{\zeta}$

入り
人

7054

 $-27-$

 $-38-$

 $\epsilon \propto \epsilon$

 $\begin{array}{l} \mathcal{R}^{\prime} \ni \\ \mathcal{R}^{\prime} \ni \end{array}$

Table XIII: UK-2 Summary of Results

J.

 $\overline{}$

9a juûû6e

UK-3: Partially Water-Filled Flask

UK-3, shown in Figure 15, represents a sealed container, partially filled with water, subject ta external heating approximating the IAEA thermal test. The external heat flux is simplified to avoid unnecessary external boundary condition complexity. The container is assumed to be sealed thereby suppressing boi.ling in the water. Natural convection is also simplified to enable relatively simple heat transfer codes to be used. Heat flow by convection is simulated by using an artificially large horizontal component of thermal conductivity for the water while the vertical component is the actual conductivity of water. In this way the effects of stratification are represented in a cost effective way using readily available codes. A sample calculation, with a fluid-flow code with three-dimensional capability, shows that the temperature predictions are of an acceptable accuracy using the anisotropic conductivity simulation.

 \bullet

 \bullet

l

The calculation is in two parts: an initial steady state is defined (in this case a uniform temperature distribution) followed by a heating transient with a constant heat flux on curved external surfaces (i.e., the lid and base are adiabatic surfaces) and finally a cool down transient when heat is rejected from the curved outer surface by radiation and convection.

The dimensions and material properties are given in Table XIV. The heat transfer characteristics are given in Table XV.

Three solutions were based on finite element models (TAU and DELFINE) and the other four on finite difference codes (HEATING-6 and SINDA). Each utilized radiation view factors for heat transfer across the nonparticipating void above the water. Constant geometry was specified, so no allowance for thermal expansions or water level changes were necessary.

The individual tabular results are given in Table XVI. The mean and standard deviations are given in Table XVII.

Figure 15. UK-3 Partially Water-Filled Flask

 $-31-$

Dimensions

The model comprises a closed cylindrical vessel, partially filled with water, with the axis vertical.

Material Properties

The uncertainty in the water surface temperature is reflected in the corresponding uncertainty in the vapor pressure which is obtained using the water surface temperature. The standard deviation is 2.4 percent of the mean surface water temperature (2[a]) but 9.5 percent of the vapor pressure. This uncertainty must be reflected in design safety margins for water-filled cavities.

Conclusions

This report contains six problems and their corresponding analyses. These problems span the ther:nal phenomena associated with internal heat generation and dissipation (US-l), a two-dimensional thermal radiation environment (US-2), phase change in a cooling medium (FR-l), fuel pin interaction (UK-l), fin heat dissipation (UK-2), and thermal stratification and vapor pressure buildup (UK-3).

7054 - 32 - 200 - 200 - 200 - 200 - 200 - 200 - 200 - 200 - 200 - 200 - 200 - 200 - 200 - 200 - 200 - 200 - 20

カンテール にんな

Intial temperature - 38°C uniform 0 to 30 minutes Uniform heat flux $q = 10$ kW/m Lid and base outside surfaces are adiabatic Natural convection coefficient: $H_w = 500 \Theta_w^{1/3} w/m^2K$ $\Theta_{\mathbf{W}} = \mathbf{T}\mathbf{s} - \mathbf{T}\mathbf{w} \mathbf{K}$ T_S = local vessel inside surface temperature (K) $T_w = local water temperature (K)$ There is no heat transfer by conduction or convection in the vapor space above the water, but radiation exchange takes place between the surfaces of the vessel and the water. $\epsilon_{\rm S}$ = 0.80 vessel surface emissivity ε_w = 1.00 water surface emissivity 30 to 60 minutes $H_E = 1.3 \Theta_E^{-1/3}$ w/m²K Convection coefficient $\varepsilon_{\rm E} = 0.80$ Surface emissivity $\Theta_{\rm E} = T_{\rm S} - T_{\rm A}$ K T_S = local vessel outside surface temperature (K) T_A = ambient temperature (311.2 K)

These problems require simulation of conduction, radiation, and a specified convection boundary. Natural convection was simulated using an anisotropic thermal conductivity.

The main thermal components of a cask were simulated including a fuel assembly as a heat source, cooling media of sodium and water, conducting cask walls, radiating gaps representing voided neutron shields, and heat dissipation fins.

The results of the analyses indicated that there are several general purpose thermal computer codes (TAU, SINDA, Q/TRAN, DELFIN'E, HEATING-6) capable of simulating cask thermal response as well as at least two special purpose codes (RIGG, COBRA) able to model fuel assembly response.

l

 \bullet

 7054 -33-

l

t,

 $\ddot{}$

 \bullet

 \bullet

 ϵ

 $\ddot{}$

 \mathbf{L}

 $\mathcal{L} = \mathcal{L} \mathcal{L}$

Table XVI: Temperatures Calculated for UK-3

 $\mathcal{O}(\mathcal{O}(\log n))$ and $\mathcal{O}(\log n)$

 λ

Table XVII: Statistical Values for UK-3

The results, when compared with analytical or experimental solutions, were within 10 percent. The intercomparison of the numerical results were also generally within 10 percent.

This set of problems provides broad coverage of the thermal phenomena of interest to cask designers and regulators. The agreement with analytical and experimental solutions, as well as the consistent results in intercomparison of codes, provides confidence that these solutions can be used in benchmarking other thermal codes.

7054 - 35

 90.00042

DISTRIBUTION: U. S. Department of Energy (226) Office of Scientific and Technical Information Attn: DOE/OSTI-4500-R74 UC-71 Oak Ridge, TN 37830 U. S. Department of Energy (2) Routing RW-33 Attn: L. Barrett W. Lake 1000 Independence S. W. Washington, DC 20585 U. S. Department of Energy (4) Attn: T. Hindman L. Harmon F. Falci R. Garrison Routing DP-123 Washington, DC 20545 Office of Security Evaluations (2) Attn: Dr. J. L. Torres C. Mauck Defense Programs, DP-4, GTN Washington, DC 20545 U. S. Department of Energy (1) Albuquerque Operations Office Attn: J. E. Bickel Post Office Box 5400 Albuquerque, NM 87115 U. S. Department of Energy (1) Albuquerque' Operations Office Attn: K. Golliher 4308 Carlisle, N. E. Albuquerque, NM 87107 U. S. Department of Energy (2) Chicago Operations Office Attn: T. Haugen J. Roberts Building 350 9800 South Cass Avenue Argonne, IL 60439 U. S. Department of Energy (1) Idaho Operations Office Attn: C. Dwight 550 Second Street Idaho Falls, ID 83401

Atomic Energy Establishment (1) UKAEA Room 337, Building 41 Attn: M. Burgess Dorchester DT2 EDH, Dorset UNITED KINGDOM British Nuclear Fuels, PLC (1) Risley Attn: E. Livesey Warrington WA3 6AS UNITED KINGDOM E. M. Systems (1) PI 457 Attn: D. Mennerdahl S-186 00, Vallentuna SWEDEN ENEA (2) TIB,'FICS Attn: A. Cherubini S. Giambuzzi Via Mazzini 2 I-40100, Bologna ITALY Commissariat A L'Energie Atomique (2) DEDR/DEMT Cen de Saclay Attn: J. Geffroy S. Bourdon 91191 Gif-sur-Yvette, Cedex FRANCE Nuclear Energy Agency (2) Organization of Economic Cooperation and Development Banque de Donnees de L'Aen Attn: P. Nagel E. Sartori 91191 Gif-sur-Yvette, Cedex FRASCE 3141 S. A. Landenberger (5) 3141-2 For DOE/OSTI (8) 3151 W. I. Klein (3) 6000 D. L. Hartley 6300 R. W. Lynch 6320 J. E. Stiegler Attn: TTC Master File 6320 TTC Library (25)

'7' ': 1 ,~,,. ,,, '.,'

6321 R. E. Luna
6322 J. M. Freedman 6322 J. M. Freedman 6322 R. E. Glass (5) 6323 G. C. Allen 8524 P. W. Dean

 $\frac{1}{2}$

 $\label{eq:2.1} \frac{1}{\sqrt{2}}\int_{\mathbb{R}^3} \frac{d\mu}{\mu} \left(\frac{d\mu}{\mu} \right)^2 \frac{d\mu}{\mu} \left(\frac{d\mu}{\mu} \right)^2$

 \bar{z}

reg7045 -37-

 $\mathcal{A}^{\mathcal{A}}$

 $\hat{\mathcal{A}}$

 $\bar{\mathcal{A}}$

J.

 \mathcal{A}^{\pm} $\sim 10^7$

 \sim

